

LA-UR-17-25633

Approved for public release; distribution is unlimited.

Title: The Monte Carlo Method and MCNP - A Brief Review of Our 40 Year

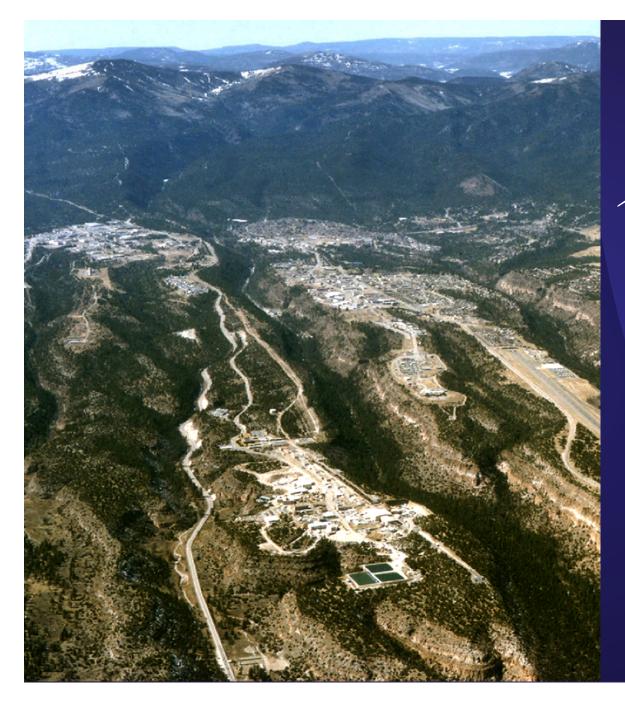
History

Author(s): Sood, Avneet

Intended for: IRRMA X, 2017-07-09 (Chicago, Illinois, United States)

Issued: 2017-07-11







Delivering science and technology to protect our nation and promote world stability

The Monte Carlo Method and MCNP – A Brief Review of Our 40 Year History

Presentation to the International Topical Meeting on Industrial Radiation and Radioisotope Measurement Applications Conference



Avneet Sood, PhD XCP-3 Group Leader

10 July 2017



Outline

- Origins of the Monte Carlo method
 - Development of electronic computers and Monte Carlo method occur simultaneously
 - Ulam, Von Neumann, Richtmeyer, Metropolis, Fermi
- Growth and usage of Monte Carlo codes
 - 1950's, 1960's, and 1970's
 - Monte Carlo becomes mainstream; nuclear criticality and reactor
- Emergence of MCNP
- MCNP's history
- MCNP's upcoming future

*nearly all references can be found at: https://laws.lanl.gov/vhosts/mcnp.lanl.gov/references.shtml

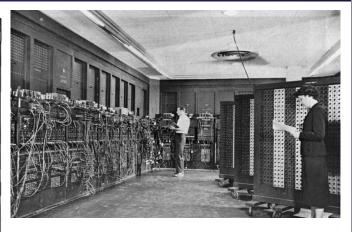
Abstract

The Monte Carlo method for radiation particle transport has its origins at LANL dating back to the 1940's. The creators of these methods were Drs. Stanislaw Ulam, John von Neumann, Robert Richtmyer, and Nicholas Metropolis. Monte Carlo methods for particle transport have been driving computational developments since the beginning of modern computers; this continues today. In the 1950's and 1960's, these new methods were organized into a series of special-purpose Monte Carlo codes, including MCS, MCN, MCP, and MCG. These codes were able to transport neutrons and photons for specialized LANL applications. In 1977, these separate codes were combined to create the first generalized Monte Carlo radiation particle transport code, MCNP. In 1983, MCNP3 was released for public distribution to the Radiation Safety Information Computational Center (RSICC). The upcoming release of MCNP (version 6.2) is expected in June 2017. Approximately 20,000 copies of MCNP have been distributed to users in government institutions, academia, and private industries worldwide. This talk will review our history, current status, and future directions.

The Origins of Monte Carlo – 1946 Stanislaw Ulam

- "The year was 1945. Two earthshaking events took place: the successful test at Alamogordo and the building of the first electronic computer" - N. Metropolis
- The method was invented by Stanislaw Ulam in 1946 playing Solitaire while recovering from an illness.
- "After spending a lot of time trying to estimate success by combinatorial calculations. I wondered whether a more practical method...might be to lay it out say one hundred times and simply observe and count the number of successful plays" - S. Ulam





ENIAC – the first electronic computer, University of Pennsylvania. Solved ballistic trajectory problems for Army Ballistics Research Lab. Used electron tubes instead of mechanical counters. Minutes instead of days. Declassified in 1946.



Trinity – code name for first nuclear detonation

"Stan Ulam, John von Neumann, and the Monte Carlo Method," R. Eckhardt, Los Alamos Science Special Issue 1987.

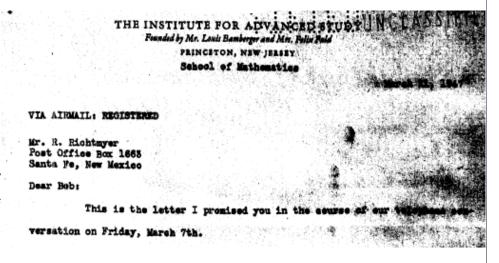
The Origins of the Monte Carlo Method

- Ulam describes this idea to John von Neumann in a conversation in 1946
- Von Neumann is intrigued
 - 1943: Electro-Mechanical computers solved nonlinear diff. eq. via production line. Punch card used for every point in space/time
 - New computers could count/arithmetic and hence solve difference equations (BRL at Aberdeen, MD)
 - Statistical sampling on electronic computers
 - Especially suitable for exploring neutron chain reactions in fission – neutron multiplication rates
- R.D Richtmyer and J. von Neumann "Statistical Methods in Neutron Diffusion", Los Alamos (LAMS-557) April 9, 1947.
 - Detailed letter from John von Neumann to Robert Richtmyer describing a conversation in March 1947
 - "I have been thinking a good deal about the possibility of using statistical methods to solve <u>neutron diffusion and</u> <u>multiplication problems</u> in accordance with the principle suggested by Stan Ulam"
 - Letter contained 81-step pseudo code for using MC for particle transport



- J. Von Neumann invented scientific computing in the 1940's
- Stored programs now called software
- Algorithms/Flowcharts
- Hardware design





The first Monte Carlo (pseudo) Code - 1947

Von Neumann's Assumptions:

- Time-dependent, continuous energy, spherical but radially-varying, 1 fissionable material, isotropic scattering and fission production, fission multiplicities of 2,3, or 4
- Suggested 100 neutrons each to be run for 100 collisions
 - Thought these were too much
- Estimated time: 5 hrs on ENIAC
- Richtmyer's response:
 - Very interested in idea and proposed suggestions
 - Allow for multiple fissionable materials, no fission spectrum energy dependence, single neutron multiplicity, run for computer time not collisions
- ENIAC: first calculations run April/May 1948
 - Code finalized in <u>December 1947</u>;
 - Continuous energy neutrons, fission spectra and XS tabulated at interval mid-points, histogram energydependence of XS, pseudo-RN.

R.D Richtmyer and J. von Neumann "Statistical Methods in Neutron Diffusion", Los Alamos (LAMS-557) April 9, 1947.

April 2, 1947

Professor John vouHeumenn, The Institute for Advanced Study, School of Mathematics Princeton, New Jersey

Dear Johnny:

As Stan told you, your letter has aroused a great deal of interest here. We have had a number of discussions of your method and Bengt Carlson has even set to work to test it out by hand calculation in a simple case.

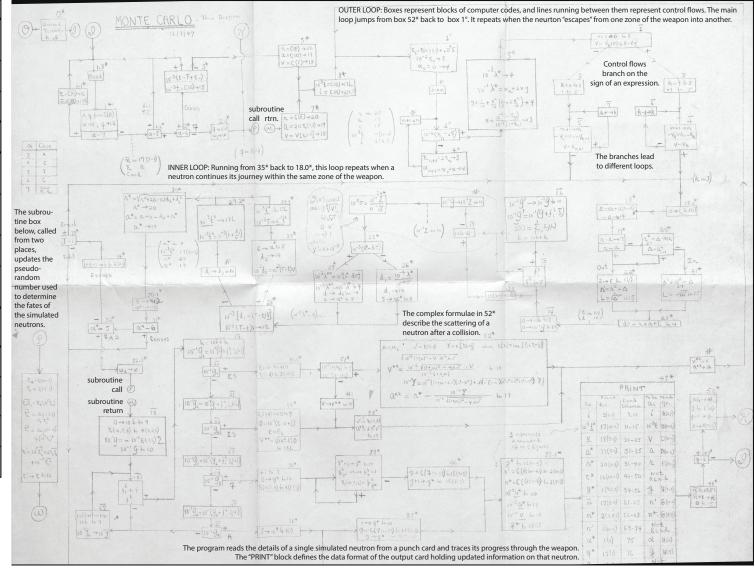
Thomas Haight, et al., "Los Alamos Bets on ENIAC: Nuclear Monte Carlo Simulations, 1947-1948, IEEE Ann. Of History of Comp July-Sept 2014

ENIAC in Action:

Boxes	Function	
1* - 8*	Read a card and store neutron characteristics	
1° - 4°	Calculaterandom parameter λ*	
1-7	Find neutron's velocity interval	
18* - 23*	Calculate distance to zone boundary	
14 - 17.1, 24*	Calculate cross- section of material In zone	
25* - 27*	Determine if terminal event is collision or escape	
28* - 30*	Determine if a census comes first	
31* - 35*	Discriminate between terminal events	
Subroutine ρ/ω	Refresh random number	
18 - 27	Determine collision type	
51* - 52*	Elastic scattering	
53* - 54*	Inelastic scattering	
36* - 39*, 46*	Absorption/fission	
37.1*, 47* - 50*	Print card and restart main loop	

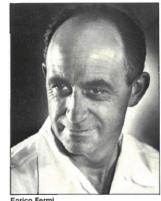


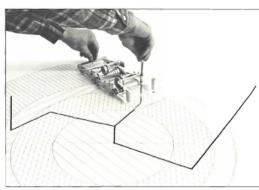
Thomas Haight, Mark Priestley, and Crispin Rope, "ENIAC in Action: Making and Remaking the Modern Computer," MIT Press 2016



Enrico Fermi: Independently developed Monte Carlo!

- Emilio Segre, Fermi's student and collaborator:
 - "Fermi had invented, but of course not named, the present Monte Carlo method when he was studying the moderation of neutrons in Rome. He did not publish anything on the subject, but he used the method to solve many problems with whatever calculating facilities he had, chiefly a small mechanical adding machine"
- Astonished Roman colleagues when he would predict experimental results remarkably accurately. He revealed that he used statistical sampling techniques whenever insomnia struck.
- 15 years prior to Ulam
- While in Los Alamos and awaiting ENIAC's move, he created an analog device to study neutron transport.
 - Called FERMIAC
 - Generated the site of next collision based upon characteristics of material; Another choice was made at boundary crossing; "slow" and "fast" neutron energies





FERMIAC





Los Alamos Scientists: Bengt Carlson, Nicholas Metropolis, LDP King with Fermiac (1966)

MANIAC – Nicholas Metropolis

- Post-war ENIAC started a revolution that continues today
- MANIAC Mathematical and Numerical Integrator and Computer
 - Was a product of Nicholas Metropolis at LANL; borrowed concepts from von Neumann's IAS, operational in 1952;
 - MADCAP high-level language and compiler
 - Rapid growth of computing: AVIDAC (Argonne)
 ORACLE (Oak Ridge), ILLIAC (U of I)
 - Special effort that helped bind Von Neumann,
 Fermi, Beta, Teller, Ulam, Feynman, etc in postwar efforts. MANIAC was a fascination.
 - First time "Monte Carlo" appears in publication:
 - Nicholas Metropolis and S. Ulam, "The Monte Carlo Method," Journal of the American Statistical Association Vol. 44, No. 247 (Sep., 1949)
 - MC on MANIAC used for multiple problems other than radiation transport:

JOURNAL OF THE AMERICAN STATISTICAL ASSOCIATION

Number 247

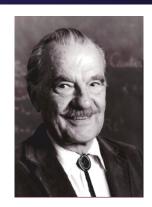
SEPTEMBER 1949

Volume 44

THE MONTE CARLO METHOD

NICHOLAS METROPOLIS AND S. ULAM Los Alamos Laboratory

We shall present here the motivation and a general description of a method dealing with a class of problems in mathematical physics. The method is, essentially, a statistical approach to the study of differential equations, or more generally, of integro-differential equations that occur in various branches of the natural sciences.







Pion-proton phase-shift analysis (Fermi, Metropolis; 1952)

Phase-shift analysis (Bethe, deHoffman, Metropolis; 1954)

Nonlinear coupled oscillators (Fermi, Pasta, Ulam; 1953)

Genetic code (Garnow, Metropolis; 1954)

Equation of state: Importance sampling (Metropolis, Teller; 1953)

Two-dimensional hydrodynamics (Metropolis, von Neumann; 1954)

Universalities of iterative functions (Metropolis, Stein, Stein; 1973)

Nuclear cascades using Monte Carlo (Metropolis, Turkevich; 1954)

Anti-clerical chess (Wells; 1956)

The lucky numbers (Metropolis, Ulam; 1956)

1950's: Monte Carlo is becoming mainstream!

- Herman Kahn, "Applications of Monte Carlo," AECU-3259 (April 19, 1954).
 - General and not specific to radiation particle transport
 - Direct sampling for common distributions
 - Rejection sampling if direct sampling does not work
 - Use a simple, easy to sample distribution, to get an estimate and correct later.
 - Russian Roulette
 - Stratified sampling, importance sampling, splitting
- E.D. Cashwell and C.J. Everett, "A Practical Manual on the Monte Carlo Method for Random Walk Problems," LA-2120 (December 18, 1957)
 - Well-described report specific to particle transport
 - Detailed diagrams and flowcharts
 - Neutron collisions (in)elastic scattering, fission, etc
 - Photon collisions Compton scattering, photoelectric, pair prouction
 - Particle direction after collision direction cosines
 - Did not deal with thermal neutron collisions nor pseudo-random number generation

Thomas Sutton and David Griesheimer, "The Monte Carlo Method: The Past 70 Years, Current State and Future Prospects," ANS Math and Computational Division Topical (Jeju, South Korea, April 2017)

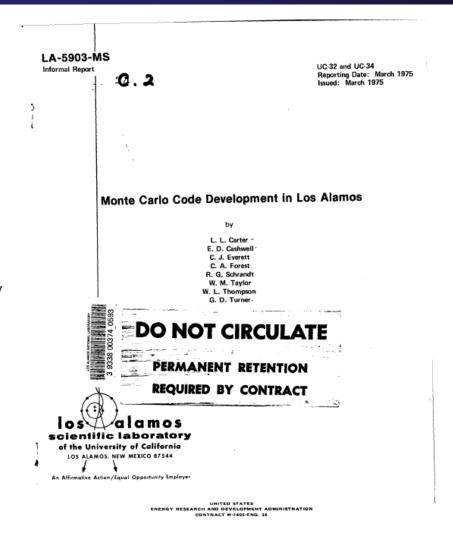
1960's: Initial work on Criticality and Reactor Calculations

- JJ.B. Parker, and E.R. Woodcock, "Monte Carlo Criticality Calculations," Prog. In Nucl. Ener, 4 (1961).
 - Introduced concept of neutron generations or batches of particles as histories
- E.R. Woodcock, T. Murphy, P.J. Hemmings, and T.C. Longworth, "Techniques Used in the GEM Code for Monte Carlo Neutronics Calculations in Reactors and Other Systems of Complex Geometry," ANL-1050 (1965).
 - Describes "Woodcock tracking" (aka delta-tracking)
 - Regular tracking on geometries with many surfaces is expensive; esp for 2nd order surfaces
 - Avoids multiple distance-to-boundary calculations by using fictitious XS and adjusting
- J. Lieberoth, "A Monte Carlo Technique to Solve the Static Eigenvalue Problem of the Boltzmann Transport Equation," Nukleonik 11, 213-219 (1968)
- M.R. Mendelson, "Monte Carlo Criticality Calculations for Thermal Reactors," Nucl. Sci. Eng 32, 319-331 (1968).

Thomas Sutton and David Griesheimer, "The Monte Carlo Method: The Past 70 Years, Current State and Future Prospects," ANS Math and Computational Division Topical (Jeju, South Korea, April 2017)

1970: MCNP first emerges

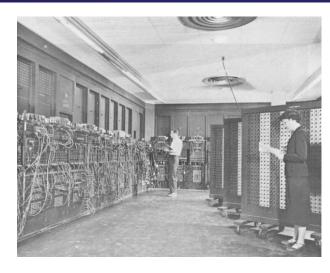
- 1974 NEA Committee on Reactor Physics (NEACRP)
 - E.D. Cashwell presented a paper on Monte Carlo development at Los Alamos
 - MCN neutrons
 - MCNA neutron adjoint
 - MCG gamma rays
 - MCP general photons
 - MCNG coupled neutron-gamma ray
 - MCMG multi-group coupled neutron-gamma ray
 - MCGE coupled electron-photon
 - MCGB gamma rays with Bremsstrahlung
- 1977: MCNG was merged with MCP to form MCNP
 - 2017 is the 40th Anniversary of MCNP
 - 2017 is also the 70th Anniversary of the Monte Carlo method



Monte Carlo & MCNP History

ENIAC - 1945

30 tons
20 ft x 40 ft room
18,000 vacuum tubes
0.1 MHz
20 word memory
patchcords



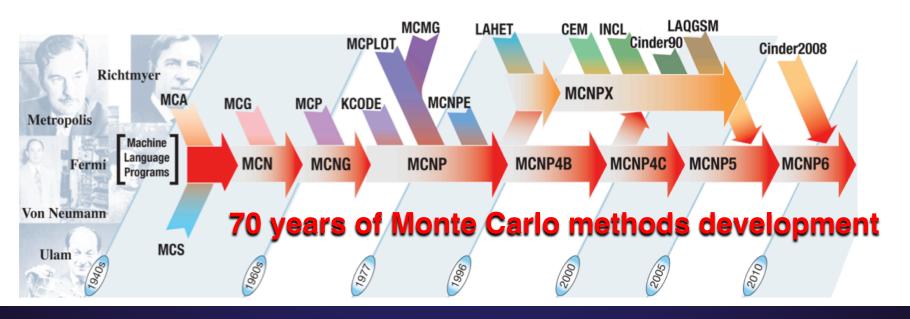
Manhattan Project – 1945...

Discussions on using ENIAC

Ulam suggested using the "method of statistical trials"

Metropolis suggested the name "Monte Carlo"

Von Neumann developed the first computer code



1980 – 1999: MCNP Becomes the Gold Standard

MCNP	Release	Some Significant New Features	
Version	Month/Year	(For a more detailed description, see each version's release notes).	
MCNP3	1983	First release through RSICC. Written in Fortran 77	
MCNP3A	1986		
MCNP3B	1988	Plotting graphics, generalized source, surface sources, repeated structures/lattice	
		geometries	
MCNP4	1990	Parallel multitasking, electron transport	
MCNP4A	10/1993	Enhanced statistical analysis, new photon libraries, ENDF-6, color X-Windows	
		graphics, dynamic memory allocation	
MCNP4B	4/1997	Operator perturbations, enhanced photon physics, PVM load balancing, cross-section	
		plotting, 64-bit executables, lattice universe mapping, enhanced lifetimes	
MCNPX 2.1.5	11/1999	First public release of MCNPX, based on MCNP4B with CEM INC, HTAPE3X,	
T Goorley et al "Initial MC		mesh and radiography tallies, and an improved collisional energy loss model.	

T. Goorley, et al. "Initial MCNP6 Release Overview," LA-UR-13-22934 (2013)

- Key Value: MCNP provides a predictive capability that can replace expensive or impossible-to-perform experiments
- Used to design large-scale measurements providing significant time/cost savings
- MCNP represents a synergistic capabilities developed at LANL
 - Evaluated nuclear data (ENDF) and data processing code NJOY
 - MCNP could not exist without this!
- International user community's high confidence in MCNP's predictive capabilities are based upon its performance with verification and validation test suites.

2000 – 2011: MCNP undergoes exponential growth

MCNP4C	4/2000	Unresolved resonance treatments, macrobodies, superimposed importance mesh,	
		perturbation, electron transport, plotter and tally enhancements	
MCNP4C2	1/2001	Photonuclear physics, interactive plotting, plot superimposed weight-window mesh,	
		weight-window improvements	
MCNPX 2.3.0	4/ 2002	LAHET 2.8 and some 3.0 extensions.	
MCNPX 2.4.0	8/2002	Update to MCNP4C, build system for Windows OS, support for Fortran 90.	
MCNP5 1.14	11/2002	Fortran 90, photonuclear collisions, geometry superimposed mesh tallies, time	
		splitting, shared memory threading with OpenMP. Mac OSX support	
MCNP5 1.20	10/2003	Increased number of detectors to 100 and number of tallies to 1000. Mostly a code	
		defect fix release.	
MCNP5 1.30	8/2004	Explicit 8-byte integers for nps > 2.1 billion, Lattice and fmesh tally enchantments.	
		Support for MPI on Mac OSX.	
MCNPX 2.5.0	4/2005	34 particle types, four light ions, mix and match nuclear data tables and model	
		physics, CEM2k, INCL4/ABLA physics models, fission multiplicity, spontaneous	
		fission sources, pulse height tallies with variance reduction, pulse height light tally,	
		coincident capture tallies, variance reduction with model	
MCNP5 1.40	11/2005	Lethargy plots, logarithmic data interpolation, neutron multiplicity distributions,	
		stochastic geometry, source entropy, mesh tally plots, new electron energy loss	
		straggling	
MCNPX 2.6.0	4/ 2008	Depletion/Burnup, heavy ion transport, LAQGSM physics, CEM03 physics, delayed	
		gamma emission, energy-time weight windows, charged ions from neutron capture,	
		spherical mesh weight windows, spontaneous photons	
MCNP5 1.51	1/2009	Photon Doppler broadening, variance reduction with pulse height tallies, annihilation	
		gamma tracking, Doppler broadening in makxsf, large lattice enhancements	
MCNP5 1.60	8/2010	Adjoint weighted tallies for point kinetics parameters, mesh tallies for isotopic	
		reaction rates, up to 100 million cells & surfaces, up to 10 thousand tallies	
MCNPX 2.7.0	4/2011	Tally Tagging, embedded sources, cyclic time bins, focused beam sources, PTRAC	
		coincidence, LLNL fission multiplicity, Receiver-operator characterization (ROC)	
		tally, NRF data in ACE libraries, triple & quadruple coincidence, LAQGSM 3.03 and	
		CEM 3.03 physics.	

2011 – today: MCNP5 & MCNPX to MCNP6

mcnp5

neutrons, photons, electrons
cross-section library physics
criticality features
shielding, dose
"low energy" physics
V&V history
documentation

New Criticality Features

Sensitivity/Uncertainty Analysis
Fission Matrix
OTF Doppler Broadening

Fission

MCNP5/X multiplicity
LLNL fission package
CGM/LLNLGAM, CGMF (soon)

mcnp6

mcnp6.1 - 2013

mcnp6.1.1 - 2014

mcnp 6.2 - 2016/2017

mcnp6

protons, proton radiography high energy physics models magnetic fields

Partisn mesh geometry Abaqus unstructured mesh

mcnpx

33 other particle types heavy ions
CINDER depletion/burnup delayed particles

High energy physics models CEM, LAQGSM, LAHET, MARS, HETC

Continuous Testing System ~10,000 test problems / day

MCNP® Capabilities

Physics:

- Continuous energy particle transport
- Neutron, photon, electron, and many more particle types

Algorithms:

- k-eigenvalue calculations
- Fixed source calculations

Recently Implemented Features:

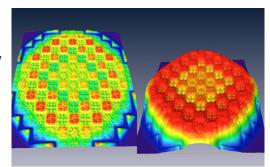
- Unstructured mesh transport
- Electric and magnetic field transport
- High-energy physics models
- 33 additional particle types
- Reactor fuel depletion and burnup
- Radiation source and detection capabilities
- Sensitivity and uncertainty analysis for nuclear criticality safety

Extensive Variance Reduction

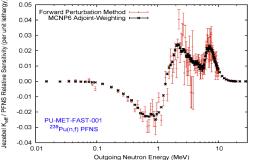
- Weight Windows
- **DXTRAN**

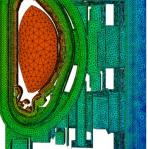
MCNP® and Monte Carlo N-Particle® are registered trademarks owned by Los Alamos National Security, LLC, manager and operator of Los Alamos National Laboratory. Any third party use of such registered marks should be properly attributed to Los Alamos National Security, LLC, including the use of the ® designation as appropriate. Any questions regarding licensing, proper use, and/or proper attribution of Los Alamos National Security, LLC marks should be directed to trademarks@lanl.gov

Whole-core Thermal & Total Flux



Jezebel - Keff Sensitivity to PFNS





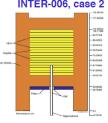
ITER Neutron Flux Calculations

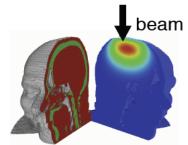
Experimental Benchmarks with Critical Assemblies

HEU-MET-THERM-003



Zeus-2, HEU-MET-INTER-006, case 2







City model used to study nuclear weapon effects

Monte Carlo Codes from across the globe

- Monte Carlo Codes Session at SNA+MC 2013
 - Annals of Nuclear Energy, 82 (2015)

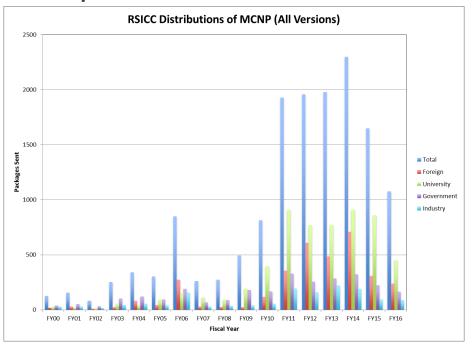
code	institution	
ARCHER	RPI	USA
COG11	LLNL	USA
DIANE	CEA	France
FLUKA	INFN & CERN	Italy & CERN
GEANT4	GEANT4	International
KENO and MONACO	ORNL	USA
MC21	Naval Nucl. Lab.	USA
MCATK	LANL	USA
MCCARD	Seoul Natl. Univ.	ROK
MCNP6	LANL	USA
MCU	Kurchatov Inst.	Russia

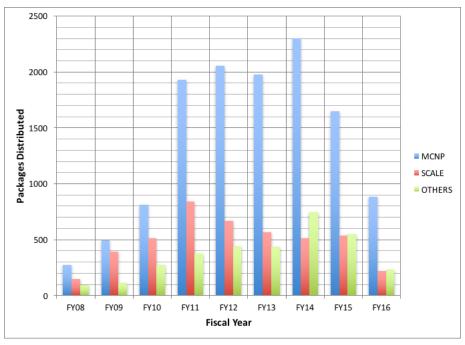
code	institution	
MONK & MCBEND	AMEC Foster Wheeler	UK
MORET5	IRSN	France
MVP2	JAEA	Japan
OPENMC	MIT	USA
PENELOPE	Barcelona Univ.	Spain
PHITS	JAEA	Japan
PRIZMA	VNIITF	Russia
RMC	Tsinghua Univ.	China
SERPENT	VTT	Finland
SUPERMC	CAS INEST FDS	China
TRIPOLI-4	CEA	France

MCNP Distribution: RSICC

- MCNP is export controlled and is distributed:
 - USA: RSICC Oak Ridge National Laboratory, LANL
 - Europe: NEA Databank; Japan, Korea: KAIST coordinated through RSICC
- Approximately 20,000 copies of MCNP licenses have been distributed.
 - 8000 copies of MCNP 6 since 2011 (Data provided by T. Valentine, RSICC)

Today: All requests eventually are through RSICC or LANL with appropriate DOE / export control reviews



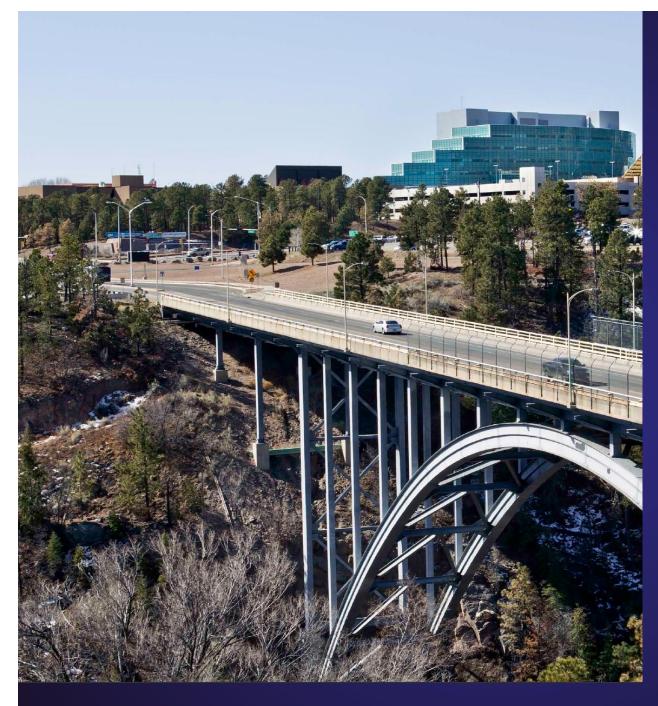


MCNP in the near future

- Motivation: LANL, DOE/NNSA, DHS-DNDO, and DTRA sponsors need a predictive capability
- Biggest needs are:
 - Validated models of geometry and materials; complex radiation sources; direct comparison with radiation detection instruments
- MCNP 2020 vision:
 - Library-based Monte Carlo framework
 - Software quality improvements

Applications:

- Next-generation high performance computers
- Multi-physics: MCNP often needed within other scientific software
- Tools to assist users
 - Geometry: Collaborations with industry
 - Allow users to take CAD/CAE, modify and develop mesh-based models; Variance reduction with Sn
 - Radiation Source: ISC generalized intrinsic source (aged) from any decay library
 - Transport physics: Correlated source/collision physics
 - Tallies: MCNP tools a package to facilitate user access to MCNP output
 - Users can produce tools to make plots, analyze data, etc without headache of having to parse data
 - One application: radiation detector response



Thank you!

Contact Info: Avneet Sood sooda@lanl.gov